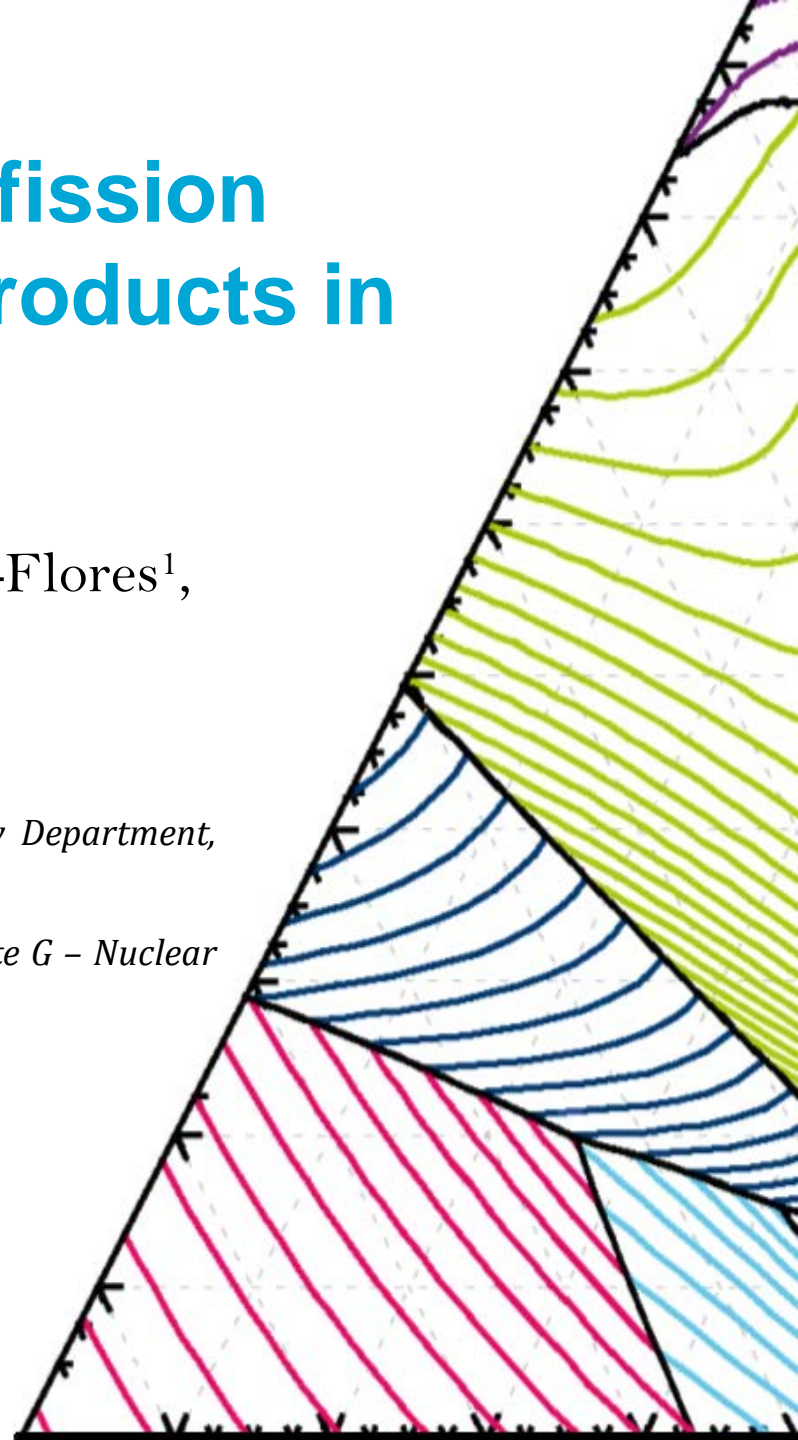


# Thermochemistry of fuel, fission products and corrosion products in Molten Salt Reactor

Elisa Capelli<sup>1</sup>, Anna L. Smith<sup>1</sup>, Jaen Ocadiz-Flores<sup>1</sup>,  
Ondřej Beneš<sup>2</sup>, Rudy J.M. Konings<sup>1,2</sup>

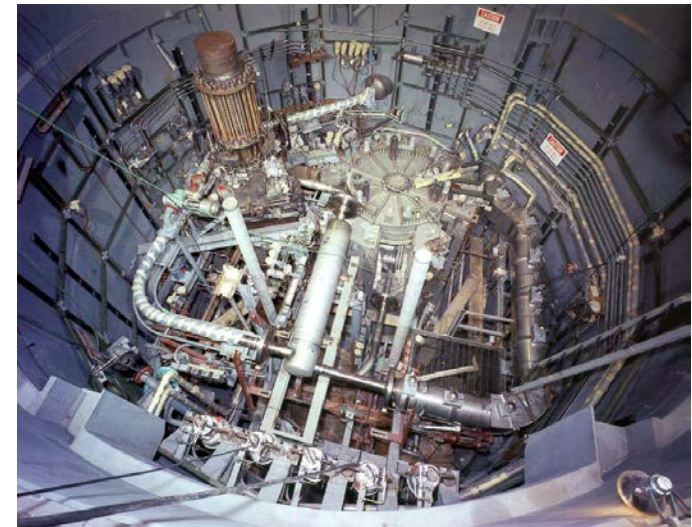
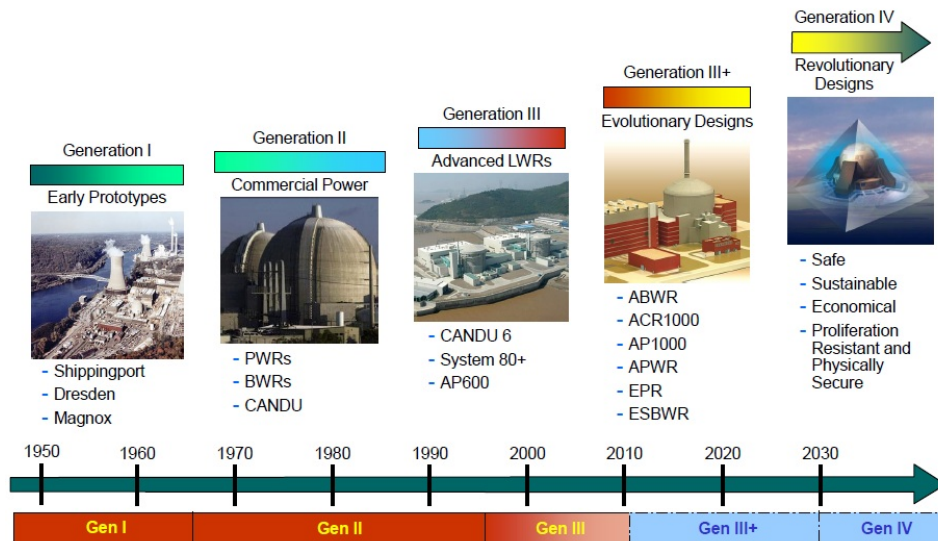
<sup>1</sup>*Delft University of Technology, Radiation, Science & Technology Department,  
Delft, The Netherlands*

<sup>2</sup>*European Commission, DG Joint Research Centre – JRC, Directorate G – Nuclear  
Safety & Security*



# Background and context of research

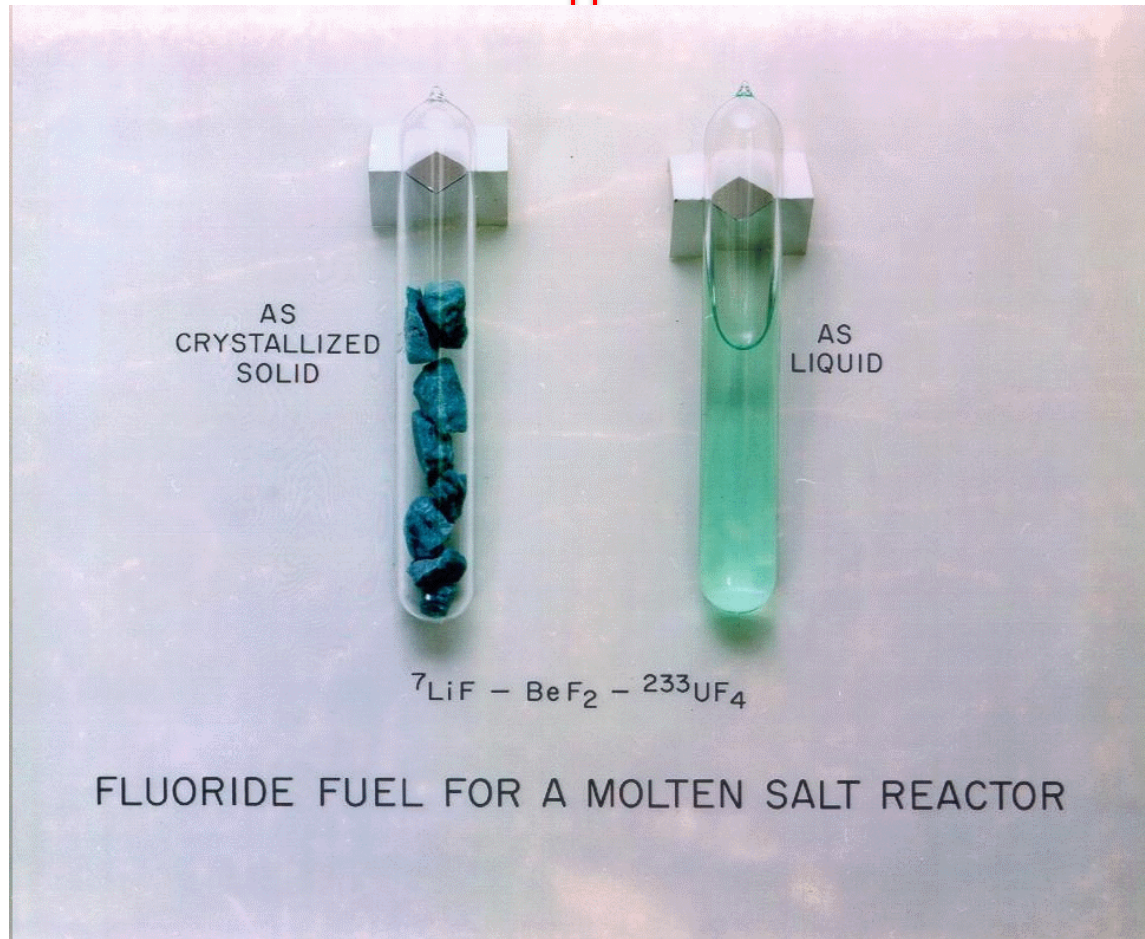
- The **Molten Salt Reactor** is among the most promising future nuclear fission reactor technologies as defined by the Generation IV International Forum. It is **safe, sustainable, economical** and **proliferation resistant**.



# Molten Salt Reactor - components

FUEL

COOLANT



# Molten Salt Reactor - Fuel

## Which are the constrains for a liquid fuel?

### ➤ NEUTRONIC PROPERTIES

- small capture cross section, moderation capability, low neutronic activation, stable under irradiation

### ➤ CHEMICAL PROPERTIES

- chemical stability, low corrosion, chemical reactivity, actinide solubility, salt clean-up

### ➤ THERMAL AND TRANSPORT PROPERTIES

- Low melting point
- Low vapour pressure and high boiling point
- Large heat capacity
- High solubility of actinides

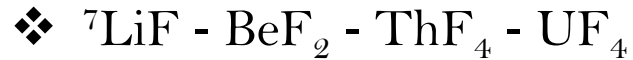
### ➤ ECONOMIC FEATURES



# Molten Salt Reactor – Fuel options

Only a limited number of compounds are suitable from neutronic considerations and **fluoride salts** are one of the best candidates.

## BASIC MIXTURE



+ additional components to improve the fuel properties if needed (NaF, KF, CaF<sub>2</sub>,...)

## CORROSION PRODUCTS

Element	Hastelloy-N (INOR-8)
Ni	Base
Cr	7.52
Mo	16.28
Ti	0.26
Fe	3.97
Mn	0.52
Nb	-
Si	0.5
Al	0.26
W	0.06
Cu	0.02
Co	0.07
Ce	-
Zr	-
B	<0.01
S	0.004
P	0.007
C	0,05

## FISSION PRODUCTS

Fission product	Yield
Kr+Xe	0.606
Lanthanides + Y	0.538
Zr	0.318
Sr+Ba	0.072
Br+I	0.015
Rb+Cs	0.004
Mo	0.201
Ru	0.126
Tc	0.059
Nb	0.014



**IRRADIATION**

# Thermodynamic modelling

**FUEL COMPOSITION  
OPTIMIZATION**

**ADDITION OF FISSION  
PRODUCTS AND CORROSION  
PRODUCTS TO THE DATABASE**

**DEVELOPMENT OF A  
THERMODYNAMIC DATABASE ON  
RELEVANT FLUORIDE SALTS**

**COMPLETE UNDERSTANDING  
OF FUEL BEHAVIOUR IN  
OPERATIONAL AND  
ACCIDENTAL CONDITIONS**

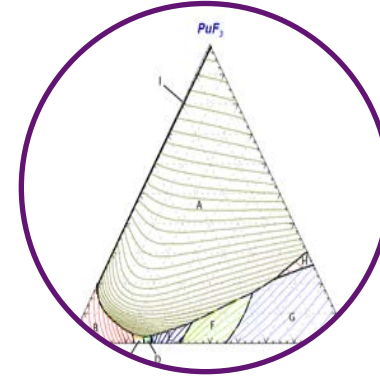


# Database development



## EXPERIMENTAL MEASUREMENTS

- Thermodynamic properties of pure compounds (heat capacity, enthalpy of fusion,...)
- Phase determination/stability
- Equilibrium data for phase diagram studies
- Vapor pressure/composition



## THERMODYNAMIC MODELING

- Assessment of binary and ternary phase diagrams
- DFT calculations
- Gas phase properties estimates



# Thermodynamic model

Binary and ternary system are assessed according to the CALPHAD approach. Three databases were developed:

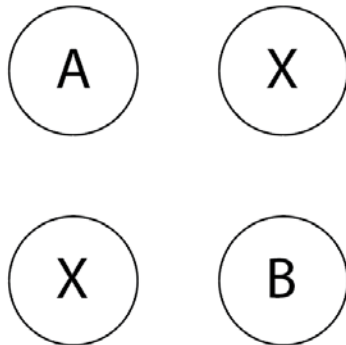
❖ Pure end-members and intermediate compounds

❖ Solid solution

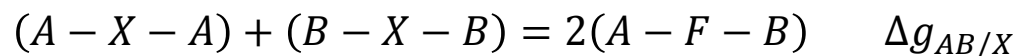
❖ Liquid solution

**Modified Quasi-chemical Model - Two sublattice Quadruplet approximation**

- Two sublattices: (A,B,...) cations and (X,Y,...) anions
- Variable composition of short range ordering
- Simultaneous treatment of FNN and SNN interactions



**FLUORIDE SYSTEMS (common-anions systems)**





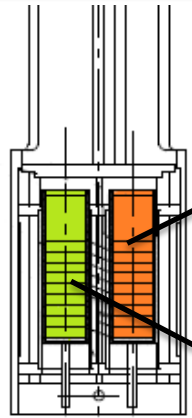
# Experimental data

## Requirements for fluorides samples:

- Purification step
- Inert atmosphere
- Encapsulation technique

## Experimental techniques:

- Differential Scanning Calorimetry
- X-ray Diffraction
- Knudsen Effusion Mass Spectrometry

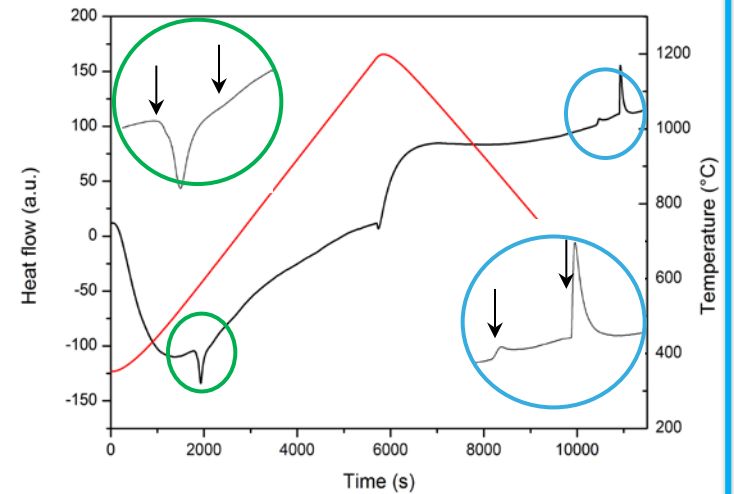


REFERENCE  
COMPARTMENT

SAMPLE  
COMPARTMENT



## Differential Scanning Calorimetry

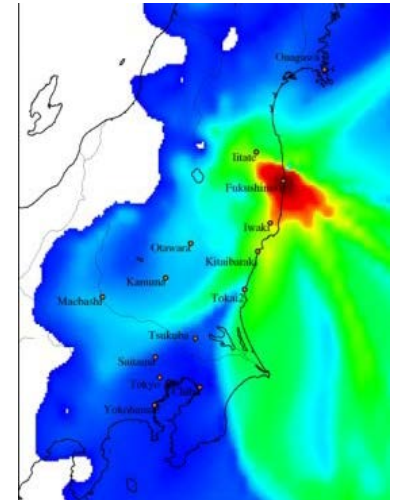


# Example of assessed systems

## ➤ Fuel salt with fission products

- LiF-ThF<sub>4</sub>-CsF-CsI system

Fission products with relatively high production yield which deserve strong attention due to their volatility and radiological effects in accidental scenarios.



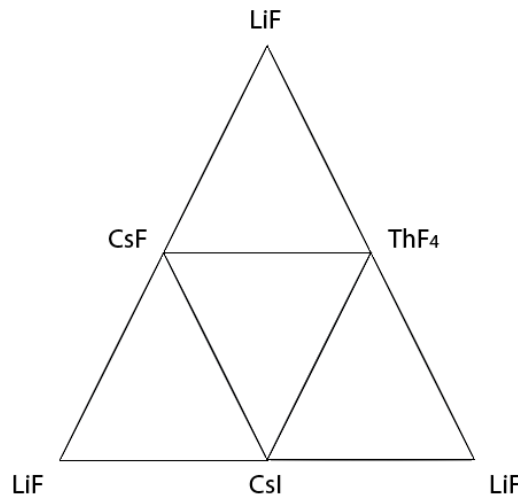
## ➤ Fuel salt with corrosion products

- LiF-NiF<sub>2</sub>, NaF-NiF<sub>2</sub>, KF-NiF<sub>2</sub> phase diagrams

# Fuel salt with fission products

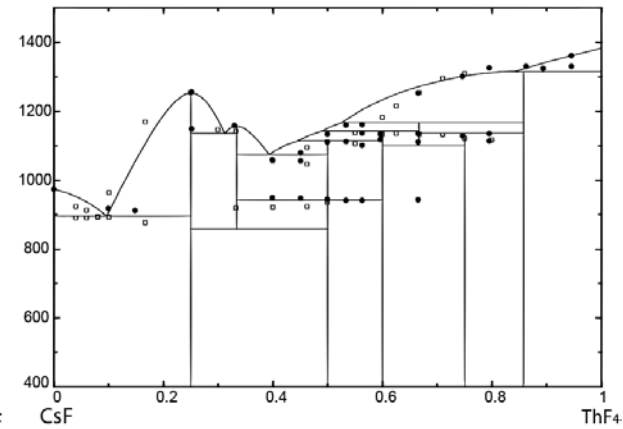
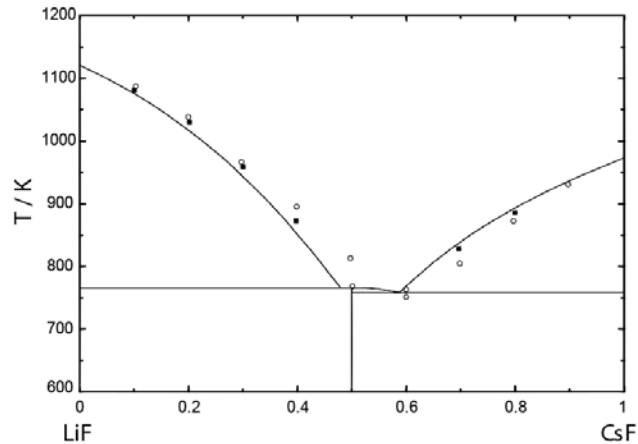
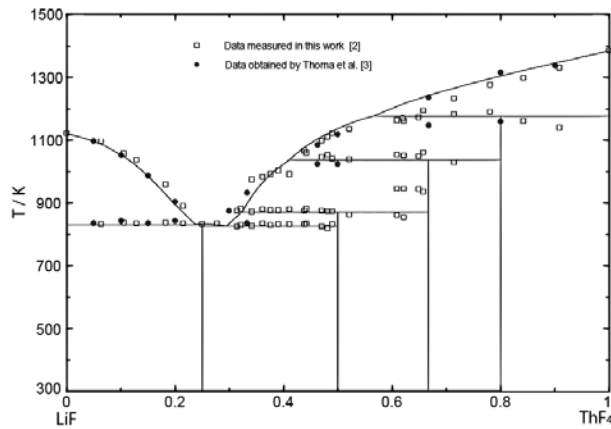
Assessment of the LiF-ThF<sub>4</sub>-CsF-CsI system:

- **CsI** is the most stable iodide in the considered mixture
- **CsF** is also formed by the cesium in excess
- Little / negligible presence of I and I<sub>2</sub>
- Little / negligible presence of Cs
- Little / negligible presence of IF<sub>x</sub> compounds



- ✓ LiF-ThF<sub>4</sub>
- ✓ CsF-ThF<sub>4</sub>
- ✓ LiF-CsF
- ✓ CsI-CsF
- ✓ LiF-CsI
- ✓ CsI-ThF<sub>4</sub>

# Binary phase diagrams – (I)



## LiF-ThF<sub>4</sub> system

- Phase equilibrium points
- Enthalpy of mixing
- LiF and ThF<sub>4</sub> activities
- Li<sub>3</sub>ThF<sub>7</sub> enthalpy of fusion

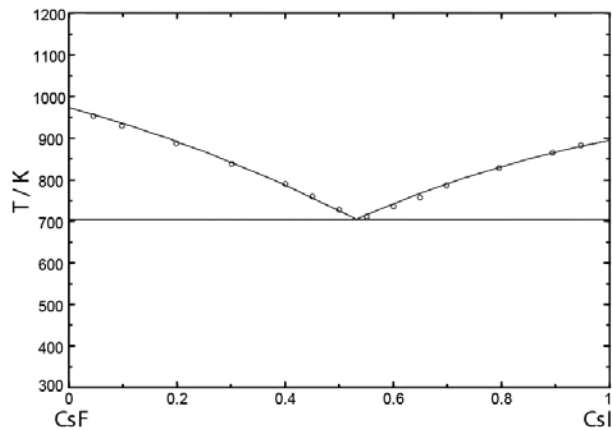
## LiF-CsF system

- Reviewed properties for CsF (*Beneš et al. 2013*)
- Phase equilibrium points (*Bukhalova 1965; Thoma et al. 1975*)
- Enthalpy of mixing (*Holm and Kleppa 1968*)

## CsF-ThF<sub>4</sub> system

- Phase equilibrium points
- Vapour pressure data
- Activity data for CsF and ThF<sub>4</sub> in the liquid solution
- Phase stability

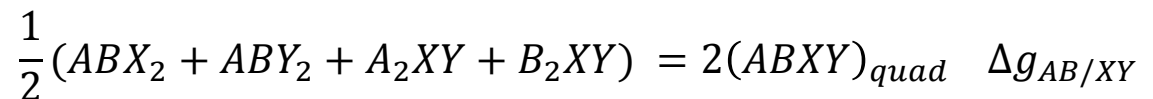
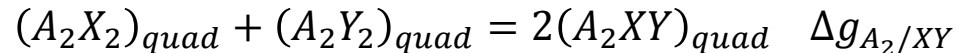
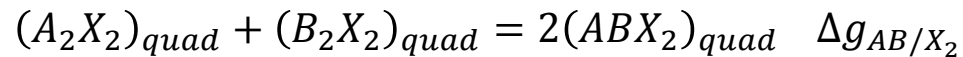
# Binary phase diagrams – (II)



## CsF-CsI system

- Phase equilibrium points  
(*Bukhalova et al. 1973*)

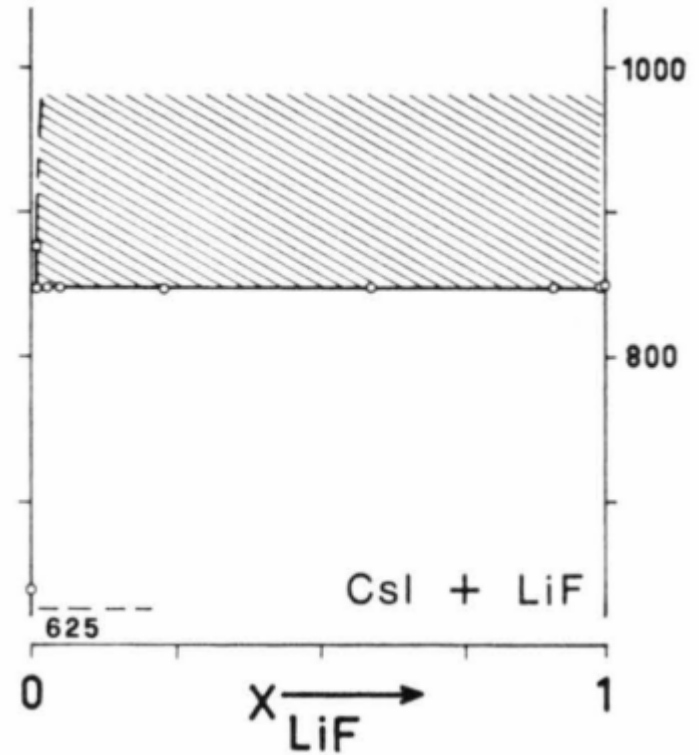
## LiF-CsI system



## ThF<sub>4</sub>-CsI system

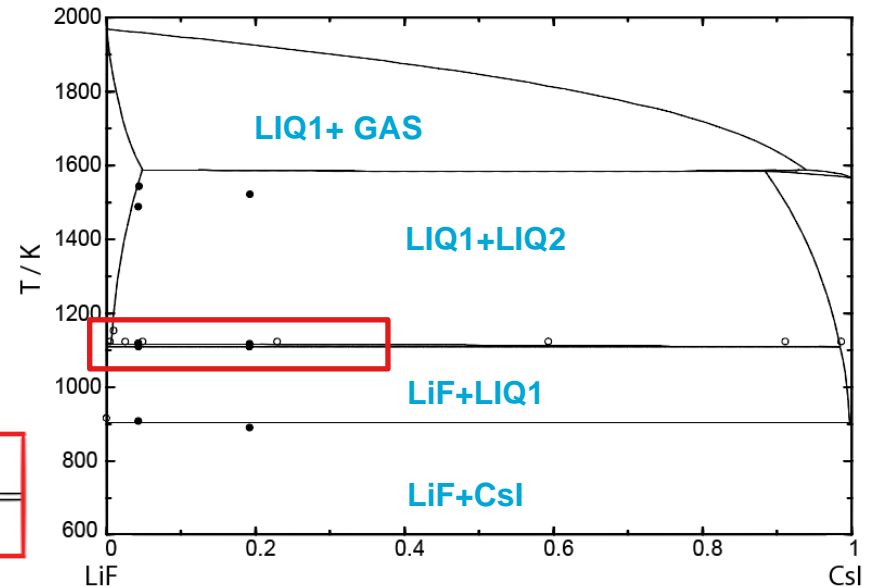
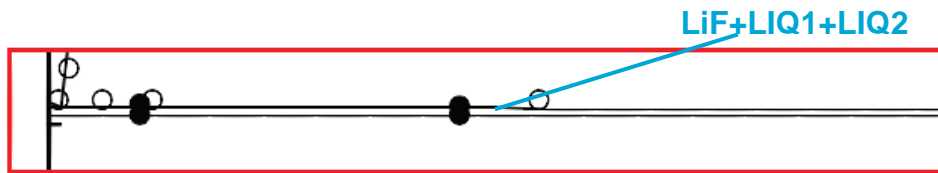
# Binary phase diagrams – LiF-CsI

- Phase equilibrium points from literature (*Margheritis, 1973*)
- Liquid-liquid immiscibility gap

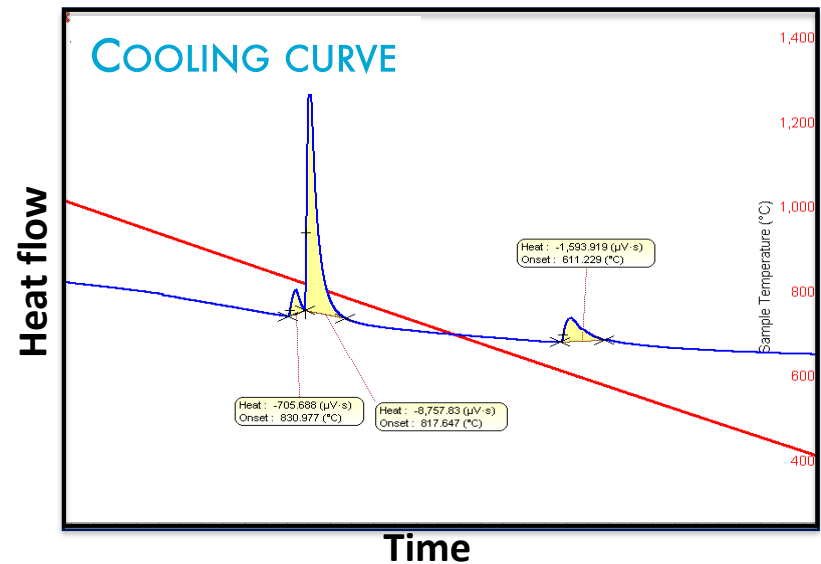
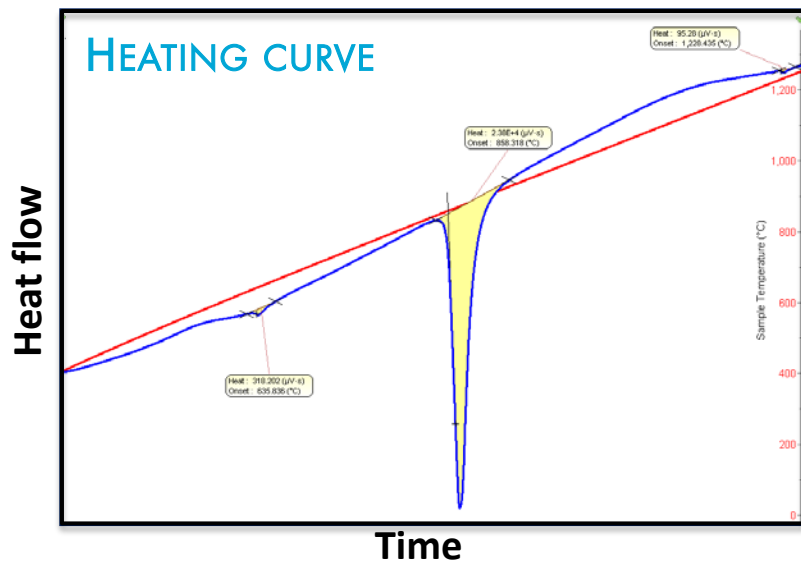


# Binary phase diagrams – LiF-CsI

- Phase equilibrium points from literature (*Margheritis, 1973*)
- Liquid-liquid immiscibility gap
- New equilibrium point

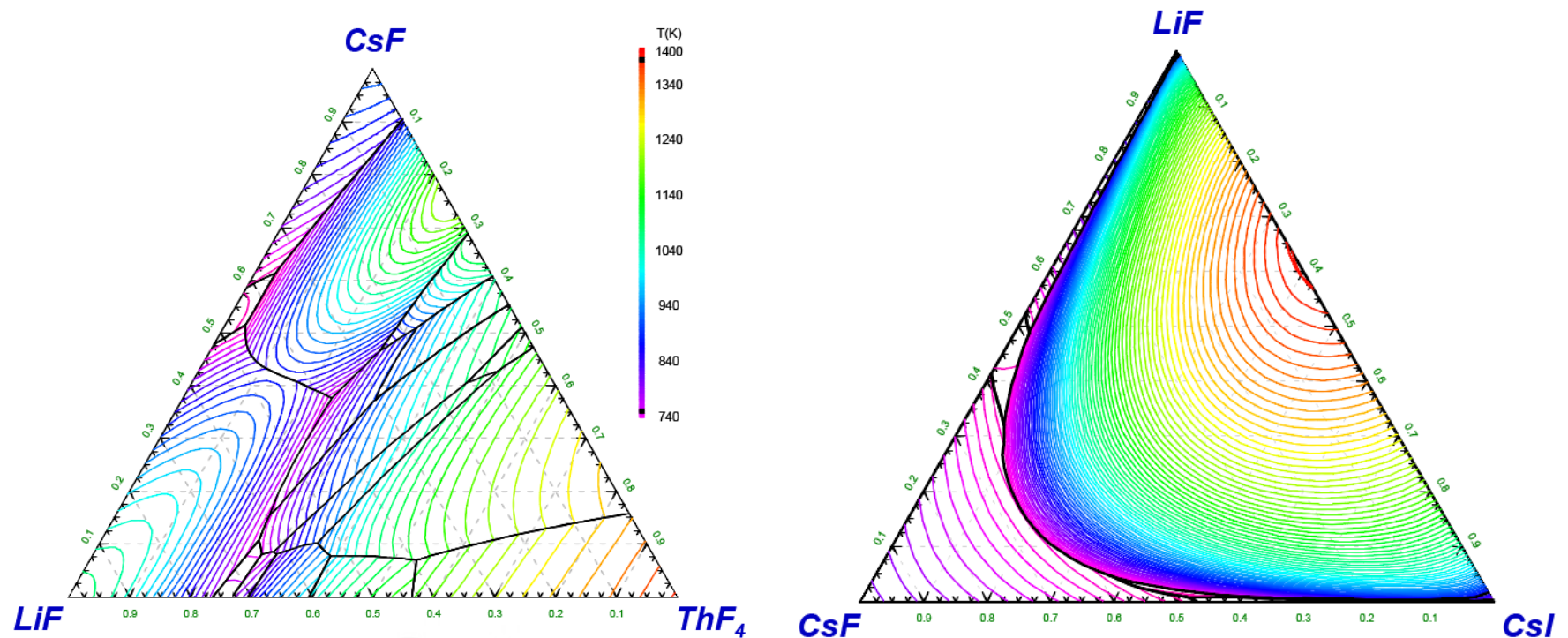


## DSC measurement – Sample LiF-CsI (xCsI = 4 mol%)



# Ternary systems

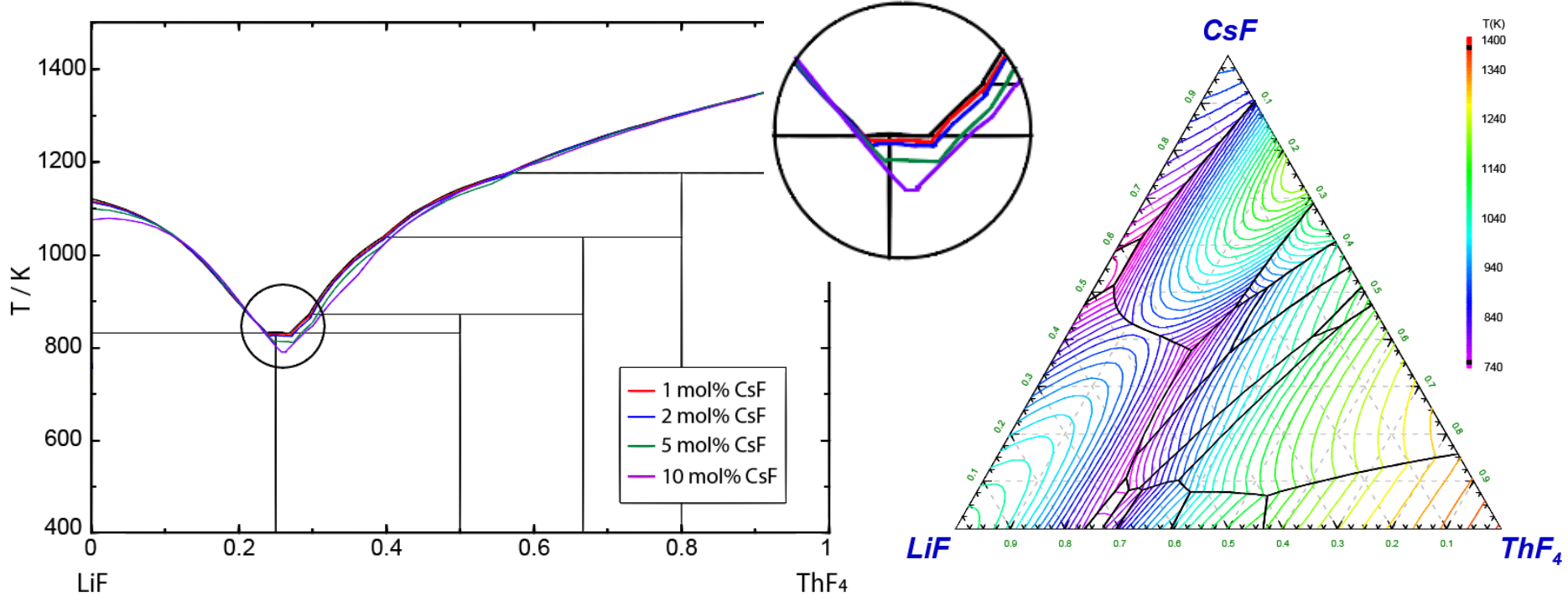
- Extrapolation using Kohler or Kohler/Topp interpolation methods.
- Small ternary parameters were introduced.



The developed database is a useful tool to predict the fuel mixture properties during operation.



# The LiF-CsF-ThF<sub>4</sub> system



Fixed LiF/ThF <sub>4</sub> ratio	Composition	T liquidus /K	P @ 900 K / Pa
	LiF-ThF <sub>4</sub> (76.2-23.8)	832 K	2,44 · 10 <sup>-3</sup>
	LiF-ThF <sub>4</sub> -CsF (75.438-23.562- <b>1.00</b> )	834 K	2,74 · 10 <sup>-3</sup>
	LiF-ThF <sub>4</sub> -CsF (7.676-23.324- <b>2.00</b> )	837 K	3,06 · 10 <sup>-3</sup>
	LiF-ThF <sub>4</sub> -CsF (72.39-22.61- <b>5.00</b> )	848 K	4,25 · 10 <sup>-3</sup>
	LiF-ThF <sub>4</sub> -CsF (68.58-21.42- <b>10.00</b> )	871 K	7,38 · 10 <sup>-3</sup>

# Example of assessed systems

## ➤ Fuel salt with fission products

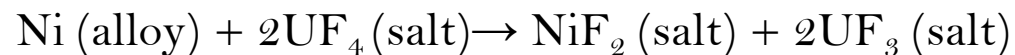
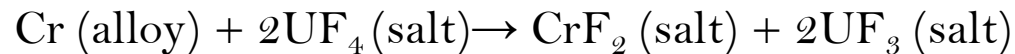
- LiF-ThF<sub>4</sub>-CsF-CsI system

## ➤ Fuel salt with corrosion products

- LiF-NiF<sub>2</sub>, NaF-NiF<sub>2</sub>, KF-NiF<sub>2</sub> phase diagrams

# Corrosion products in MSR

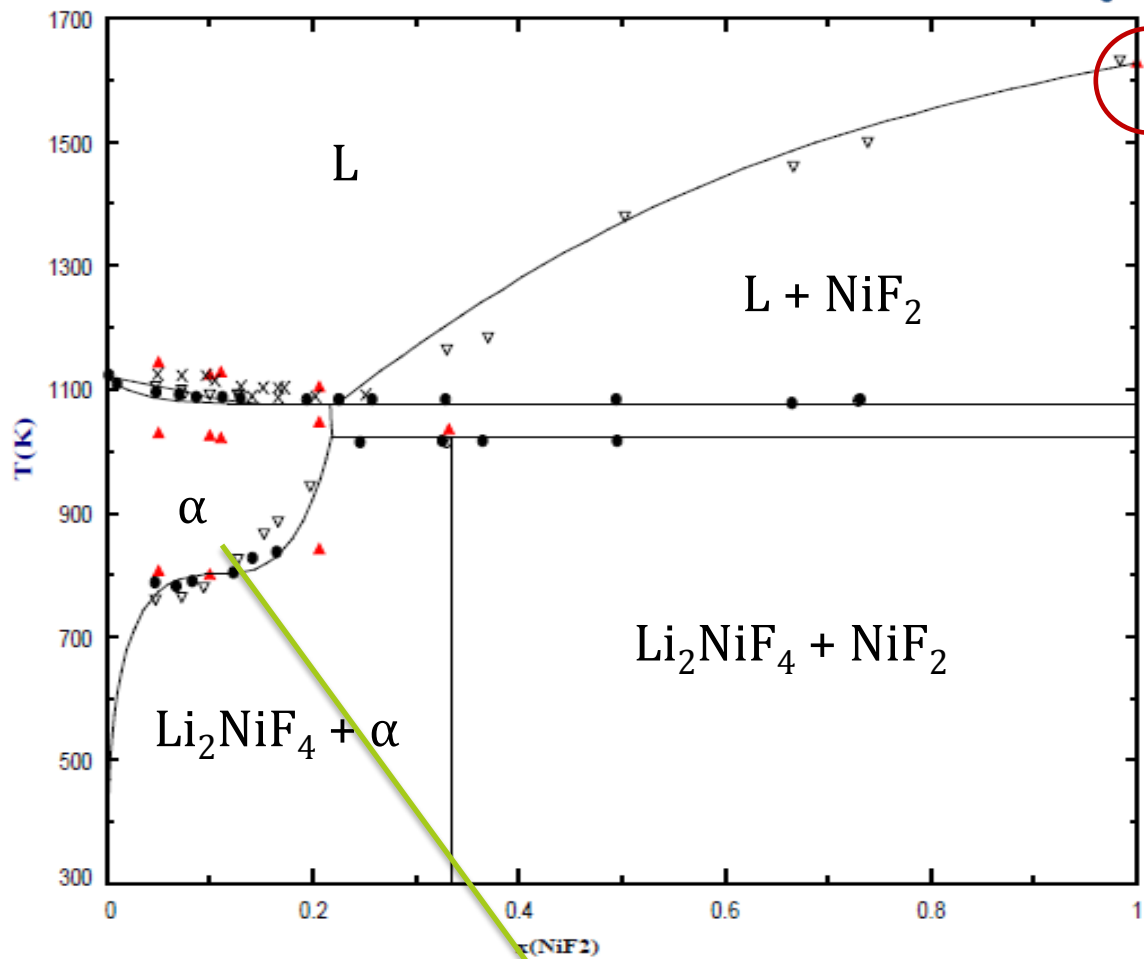
- **Ni-based alloys** are found to withstand harsh operational conditions in MSR (high temperatures, contact with corrosive salts)
- **The redox potential** of the salt is a key parameter for corrosion issues and it is controlled by the **UF<sub>4</sub>/UF<sub>3</sub> ratio**.
- However during **irradiation**, this ratio increases and corrosion products can be dissolved in the mixture:



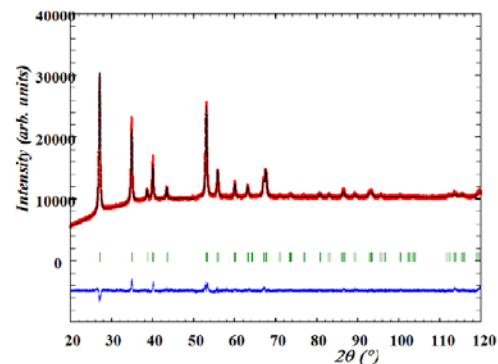
**Study of the phase equilibria of fuel salt with corrosion products as a function of temperature and salt composition**

# The LiF-NiF<sub>2</sub> system

FactSage™



Pure NiF<sub>2</sub> synthesis  
 $T_{\text{fus}} = (1628 \pm 5) \text{ K}$

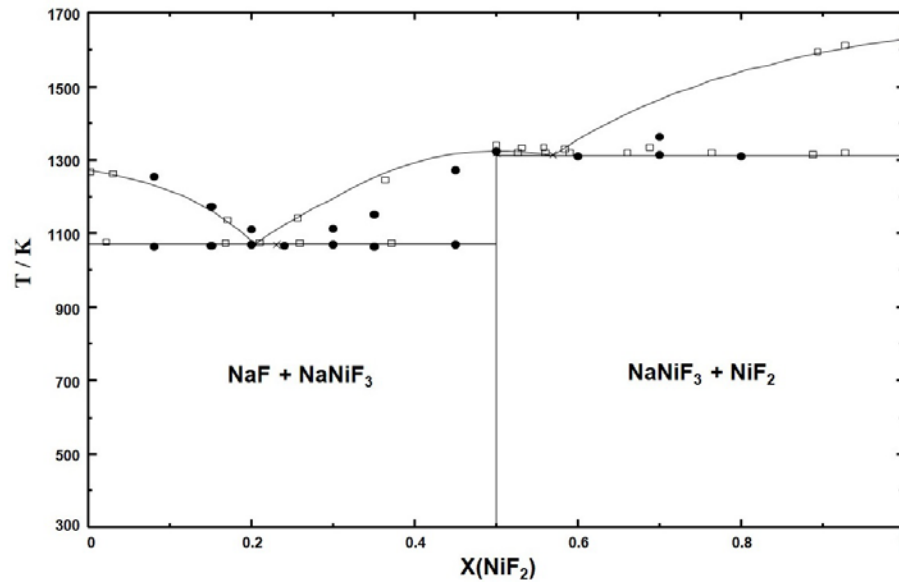


(b) XRD

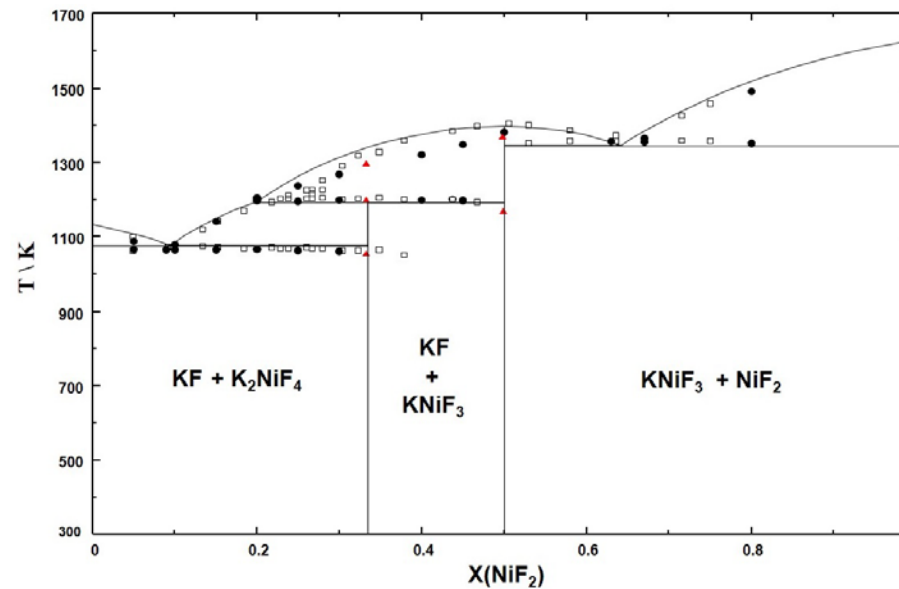
👉 Solid solution  $\alpha = (\text{Li}_{1-2x}\text{Ni}_x\text{F})$

# The NaF-NiF<sub>2</sub> and KF-NiF<sub>2</sub> systems

- ▲ This work
- Kalinnikov et al 1999.
- Petrov et al, 1952.
- × Thoma et al., reproduced from Mamantov and Smith, 1975.



- ▲ This work
- Kalinnikov et al, 1999.
- Wagner and Balz, 1952.



# Conclusion

- The generation of a reliable and consistent thermodynamic model is very important for multi-components system, such as the molten salt fuel mixture.
- A database containing the most relevant fluoride salts, including fission products and corrosion products is being developed in collaboration between the Joint Research Centre - Karlsruhe and Delft University of Technology.
- It is a powerful tool which allows the optimization of the fuel composition for MSR fuels and the prediction of their thermodynamic properties and behaviour during irradiation.

Thank you

